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January 15, 1986

TMI-2 Cleanup Project Directorate
Attn: Dr. W. D. Travers
Director
US Nuclear Regulatory Commission
c/o Three Mile Island Nuclear Station
Middletown, PA 17057

Dear Dr. Travers:

Three Mile Island Nuclear Station, Unit 2 (TMI-2)
Operating License No. DPR-73
Docket No. 50-320
Auxiliary and Fuel Handling Building Decontamination Schedule

In accordance with your letter dated March 7, 1984, the following status of the Auxiliary and Fuel Handling Building Decontamination schedule for the fourth quarter of 1985 is provided. This update lists only those areas where Technical Specification Surveillances are not being performed due to ALARA considerations.

Make-up Filter Cubicle

ALARA exemption is being taken for Technical Specification Surveillance 4210-SUR-3775.02 (4331-R3), "Fire Barrier Penetration Fire Seal Inspection".

During this past quarter, the cubicle remained inaccessible due to the location of portions of the Make-up and Purification Cesium Elution Process System. The radiation dose rate for the general area is 800 mr/hr with hot spots up to 1.25 R/hr.

Software has been prepared to remove the make-up elution equipment. It is expected that the elution equipment will be removed early in the first quarter of 1986. This action will be followed by detailed

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characterization of the cubicle. Planning and engineering are currently underway to perform both external surface and system flushing in order to reduce the dose rates.

Make-up Pump Suction and Discharge Valve Alley

ALARA exemption is being taken for Technical Specification Surveillance 4210-SUR-3775.02 (4331-R3) and 4210-SUR-3775.01, (4331-A1, "Fire Barrier Penetration Fire Seal Inspections".

General area radiation dose rates for this cubicle range from 100 mR/hr in the suction alley to 1.5-5 R/hr in the discharge alley.

During the past quarter, these cubicles were characterized using gamma spectrometry instruments to detect the presence of fuel material. Analysis of the data is not yet complete; however, it is expected to be completed during the first quarter of 1986.

In addition to the characterization performed during the past quarter, engineering specific to both external surface and internal system decontamination was completed. The actual implementation of this activity is currently pending the results of the characterization efforts and is not scheduled to begin until the second quarter of 1986.

Seal Injection Cubicle

ALARA exemption is currently being taken for Technical Specification Surveillance 4210-SUR-3244.01 (4301-M8), "Containment Integrity Verification."

During the past quarter, no additional decontamination activities were performed in this cubicle due to high radiation levels. The gamma radiation dose rate for the general area is 25 R/hr with hot spots up to 110 R/hr.

Extensive efforts have been devoted to the development of implementing software and procurement of equipment to support a comprehensive decontamination program for this cubicle. The field work will commence in the first quarter of 1986 with the installation of a supplemental ventilation system for the cubicle, installation of additional video cameras and lighting, and installation of other support systems. The actual decontamination work is scheduled to begin late in the first quarter with debris removal and gross flushing.

305' Elevation Make-up Valve Alley

ALARA exemption is being taken for Technical Specification Surveillance 4210-SUR-3775.02 (4331-R3), "Fire Penetration Fire Seal Inspection."

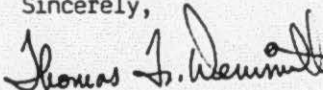
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The general area radiological dose rate is approximately 400 mR/hr. A few hot spots, which average 2-3 R/hr, remain in the piping and related components. A hot spot of 20 R/hr exists at the floor drain.

During the past quarter, the engineering required to flush much of the piping in this cubicle was completed. The flushing is planned to be performed once the Make-up Filter Cubicle is accessible. Additional engineering is currently underway to evaluate the disposition of the block orifice and related bypass piping which present the greatest radiological concern. The only field work currently planned for the first quarter is a detailed characterization using gamma spectrometry instruments.

We will continue to keep you apprised of progress in the decontamination of the Auxiliary and Fuel Handling Building.

Sincerely,



F. R. Standerfer
Vice President/Director, TMI-2

FRS/CJD/eml

Attachment